

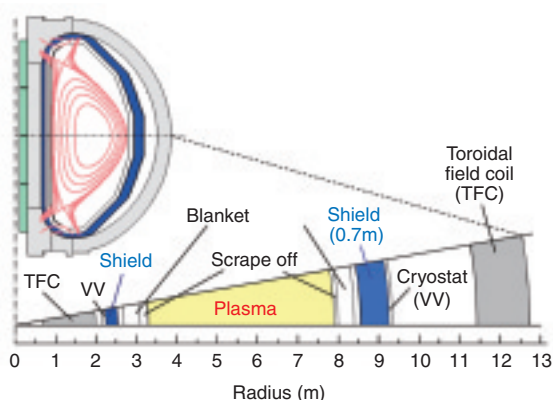
## 3-12 Compact Fusion Reactor Using Advanced Shield Materials

### — Application of Hydrogen-Rich Hydride to the Shield of Fusion Reactor —

**Table3-1 Hydrogen density of various materials**

The anticipated hydrogen concentration of  $\text{Mg}(\text{BH}_4)_2$ , which is a new candidate shielding material, is as high as  $1.32 \times 10^{29}$  H-atoms/ $\text{m}^3$ , surpassing those of already known  $\text{TiH}_2$  ( $9.1 \times 10^{28}$  H-atoms/ $\text{m}^3$ ), polyethylene and water.

Material	Appearance	Mass number	Density ( $10^3\text{kg}/\text{m}^3$ )	H density ( $10^{28}/\text{m}^3$ )
$\text{Mg}(\text{BH}_4)_2$	powder	53.99	1.48	13.2
$\text{TiH}_2$	powder	49.88	3.77	9.1
$\text{ZrH}_2$	powder	93.24	5.6	7.2
Polyethylene	Solid	14.03	0.90	7.7
$\text{H}_2\text{O}$	liquid	18.02	1.00	6.7



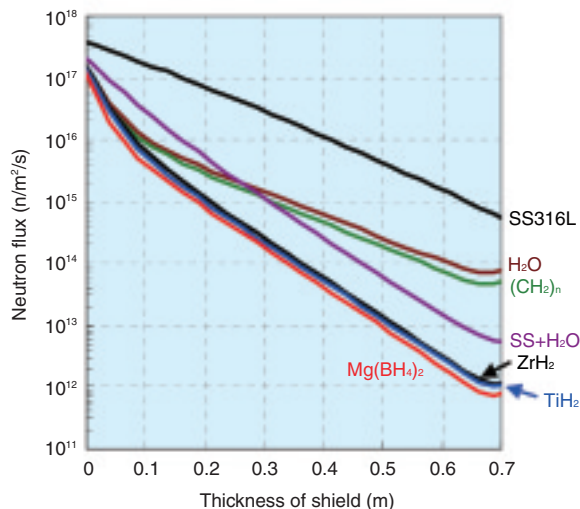
**Fig.3-26 Cross section and 1-D calculation model of a low aspect ratio tokamak reactor**

Neutron transport calculations of the 0.7 m-thick outboard shields were performed in order to evaluate the neutron shielding capability.

To demonstrate economical power generation with a compact reactor, the fusion DEMO studies at JAEA focus on a low aspect ratio ( $A$ ) tokamak. It is important to reduce the nuclear heating caused by both neutron and gamma interactions in order to maintain the superconductive state of toroidal field coils (TFC); this is done with a compact neutron shield. Such an excellent shield also can be used to protect outer structural materials from serious activation, and can lead to a dramatic reduction of radwaste.

This paper presents for the first time a new candidate neutron shielding material, magnesium borohydride ( $\text{Mg}(\text{BH}_4)_2$ ), which is one of the most promising materials for storage of large amounts of hydrogen. Table3-1 gives several metal hydrides, borohydride and their properties. The anticipated hydrogen concentration of  $\text{Mg}(\text{BH}_4)_2$  is as high as  $13.2 \times 10^{28}$  H-atoms/ $\text{m}^3$ . It is notable that some hydrides have considerably higher hydrogen content than polyethylene or water.

Neutronics calculations were carried out in order to assess the capability of  $\text{Mg}(\text{BH}_4)_2$  and metal hydrides such as



**Fig.3-27 Attenuations of fast neutron fluxes in 0.7 m-thick shields made from various materials**

The hydrogen-rich hydrides show superior neutron shielding capability compared to the conventional materials. Neutron transport calculations of the 0.7 m-thick outboard shields indicated that  $\text{Mg}(\text{BH}_4)_2$ ,  $\text{TiH}_2$  and  $\text{ZrH}_2$  can reduce the thickness of the shield by 23%, 20% and 19%, respectively, compared to the combination of steel and water.

titanium hydride ( $\text{TiH}_2$ ) and zirconium hydride ( $\text{ZrH}_2$ ) which are candidates for the new shield material. Fig.3-26 shows the poloidal cross section and a  $10^\circ$  sector model of the low aspect ratio tokamak.

Fig.3-27 shows the calculated attenuation of fast neutron ( $E > 0.1$  MeV) flux in outboard shields made from various materials. The mixture of SS and  $\text{H}_2\text{O}$  consisting of 70% stainless steel SS316L and 30% water, which is presently one of the main shielding materials in fusion device design such as ITER, also was calculated. The thickness of a shield of  $\text{Mg}(\text{BH}_4)_2$ ,  $\text{TiH}_2$  and  $\text{ZrH}_2$  can be respectively 23%, 20%, and 19% less than the combination of SS and water. Also, though the  $\gamma$ -ray shielding capability of only  $\text{Mg}(\text{BH}_4)_2$  is low, mixing  $\text{Mg}(\text{BH}_4)_2$  with steel significantly improves gamma-ray shielding.

The hydrogen-rich hydrides exhibit superior neutron shielding capability. Neutron transport calculations of the 0.7 m-thick outboard shields indicate that some hydrides allow the shield to be more than 20% thinner than the combination of SS and water.

#### Reference

Hayashi, T. et al., Neutronics Assessment of Advanced Shield Materials using Metal Hydride and Borohydride for Fusion Reactors, Fusion Engineering and Design, vol.81, 2006, p.1285-1290.