

5-8 Mass Transport Retardation Studies under In Situ Conditions — Radionuclide Migration Experiments in a Rock Fracture at a Depth of 240m —



Fig.5-17 Forklift transporting granite block containing natural fracture, quarried at a depth of 240 m

The blocks were excavated from the natural fracture zone using a diamond wire saw. Care was taken to prevent ingress of cutting debris into the fracture during the excavation and to preserve the in situ geochemical conditions by making sure that there was a positive flow of water out of the formation. Injection and withdrawal ports were installed around the perimeter of the block.

In the Japanese program, high-level radioactive waste (HLW) is vitrified, encapsulated in a metal container called overpack, surrounded by engineered buffer material, and emplaced in a repository constructed in stable rocks at a depth of 300m or greater. In safety assessments of this disposal method, the possibility that long-lived radionuclides may be leached from the wastes and may subsequently be transported through surrounding rock masses must be considered. It is therefore necessary to understand the transport of radionuclides through water-bearing fractures in rocks surrounding the repository. For this purpose, in situ radionuclide migration experiments were performed at the 240-m level in Atomic Energy of Canada Ltd.'s (AECL) Underground Research Laboratory under a five-year cooperative research program with AECL.

Two granite blocks, each with a volume of $\sim 1\text{m}^3$ and containing a single fracture were excavated from a water-bearing fracture zone with special care to minimize changes in the geochemical conditions (Fig.5-17). Migration experiments were performed by injecting ^3H , ^{85}Sr , $^{95\text{m}}\text{Tc}$, ^{237}Np , ^{238}Pu , and synthetic colloids, followed by groundwater injection.

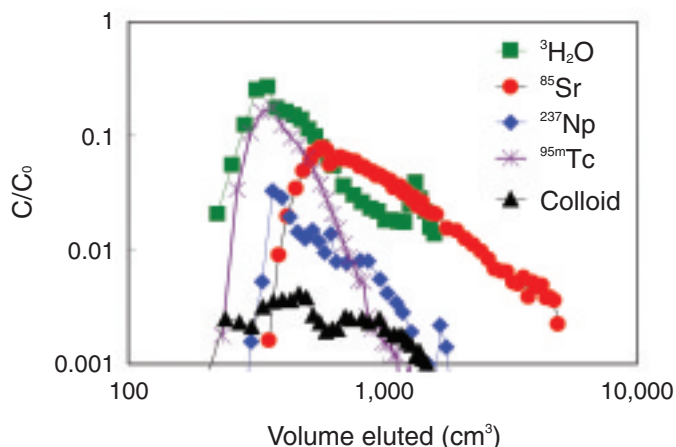


Fig.5-18 Radionuclide and colloid elution profiles, normalized to the injection concentrations

Transport of ^{85}Sr in the fracture was retarded relative to the $^3\text{H}_2\text{O}$ by diffusion into granite matrix and adsorption onto mineral surfaces. Plutonium was sorbed near the inlet to the fractures and was not detected in any of the eluted groundwater. The colloid elution exhibited a broad and low peak.

As predicted from earlier sorption studies, radionuclide transport by groundwater through the fractures was retarded by sorption on rock-forming minerals and was element-specific (Fig.5-18). Strontium exhibited weak reversible sorption. Plutonium was strongly sorbed near the inlet of the fractures, as shown by post-experiment radiometric analysis of the fracture surfaces. The elution peaks for $^{95\text{m}}\text{Tc}$ and ^{237}Np suggested that fractions of these radionuclides were transported without retardation and that the remainder was retained within the fractures. Transport of $^{95\text{m}}\text{Tc}$ and ^{237}Np can be explained by a kinetically slow redox reaction. The redox kinetics is often affected by microbial activities.

Colloid concentrations in the eluted groundwater were low, again as expected, and may have been controlled by sedimentation and diffusion into stagnant zones.

We demonstrated the retardation of radionuclide migration deep underground and gave possible mechanisms. Further investigations are needed to determine if the postulated mechanisms are valid over long periods of time, to be confident of the safety of the disposal. Of particular interest is whether colloids enhance or limit radionuclide migration in fractures.

Reference

Yamaguchi, T. et al., Radionuclide and Colloid Migration Experiments in Quarried Block of Granite under In-Situ Conditions at a Depth of 240m, Proceedings of 15th International Conference Nuclear Engineering (ICONE15), Nagoya, Japan, 2007, ICONE15-10374, in CD-ROM.