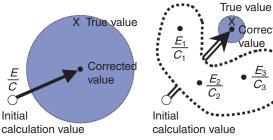
Validation of Accuracy of Prediction of Neutronic Behavior in a Design without Full-Scale Mock-Up Critical Experiments

— Development of a Prediction Uncertainty Evaluation Method Using Results of Critical Experiments —



E: Experimental value

C: Experiment analysis value

(a) Conventional method

calculation value E_i: Experimental value

Corrected

(the i-th experiment) C_i: Experiment analysis value (the i-th experiment)

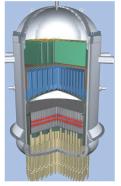
(b) Extended bias factor method

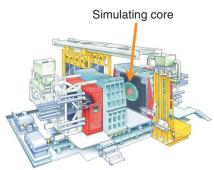
Fig.7-3 Characteristics of extended bias factor method

The conventional bias factor method which utilizes a single experiment is inaccurate if the facility of that experiment is different from the mock-up facility. The extended bias factor method which has been created constructs a facility similar to the mock-up from a number of experiments at existing facilities.

To develop a new type reactor, accurate prediction of core performance is required. This is done with calculation codes to analyze neutron behavior based on evaluated nuclear data. With a full-scale mock-up experiment very similar to the planned core, the prediction accuracy can be validated by a comparison between the analysis value of the simulation model and the experimental value. However, comparisons are often limited to experiments with existing experimental facilities. Therefore, they may not properly simulate the planned core and it is difficult to validate the prediction accuracy. However, a full-scale mock-up experiment with new experimental facilities is costly. Thus, it is very difficult to conduct full-scale mock-up experiments.

To overcome such difficulties, we created a new method named the extended bias factor method. The method can evaluate the prediction uncertainty without a full-scale mockup experiment, based on an original concept: utilize experimental results from a number of existing facilities in place of a mock-up facility. Taking the variance of experimental results into consideration, it utilizes the covariances arising from the nuclear data and the analysis method in the design values and the experiment analysis values of the planned core and each of the utilized existing cores which are caused by cross section and analysis method errors, and maximizes the correlation between the planned core and the experimental results. By this method, the





(c) Innovative light water reactor (FLWR) (d) Fast critical assembly (FCA)

Fig.7-4 Application of FCA critical experiments to a new type reactor core design

An innovative light water reactor (FLWR) is a new type reactor which will breed fuel due to a greatly changed coolant void fraction in the core. We systematically obtained critical experimental data at FCA for the FLWR cores with various void conditions, and applied the extended bias factor method, using this FCA data to evaluate FLWR designs. We could evaluate the core performance more accurately than the conventional

prediction accuracy can be properly evaluated and improved (Fig.7-3).

To apply the extended bias factor method to a real group of experimental results, we developed new methods to practically evaluate covariance between experimental values and between experiment analysis values and constructed a system to evaluate covariance between different experimental results and between different cores. Based on this system, we applied this method to a new type reactor core design using the experimental results systematically measured with several simulating cores constructed in an existing facility, Fast The application Assembly (FCA). demonstrated that the method is valid for the evaluation and improvement of the prediction accuracy of core performance of the target core (Fig.7-4).

By evaluating sources of error in prediction with the extended bias factor method, we can identify necessary new critical experiments, and the experiments and the analysis method by which uncertainties can be reduced. The present R&D results can be utilized to contribute to planning of effective R&D, saving time and cost for development of the new type reactor.

The present R&D was honored with an award for distinguished technology by the Atomic Energy Society of Japan in March, 2009.

Reference

Kugo, T. et al., Prediction Accuracy Improvement of Neutronic Characteristics of a Breeding Light Water Reactor Core by Extended Bias Factor Methods with Use of FCA-XXII-1 Critical Experiments, Journal of Nuclear Science and Technology, vol.45, no.4, 2008, p.288-303.